

Standards Manager Web Standards List
ANS-American Nuclear Society

Id	Number	Title	Year	Organization	Page
1	8.3	Criticality accident alarm system	2022	ANS	
2	8.7	Nuclear Criticality Safety in the Storage of Fissile Materials	2022	ANS	
3	RA-S-1.1	Standard for Level 1/Large Early Release Frequency Probabilistic Risk Assessment for Nuclear Power Plant Applications	2022	ANS	
4	RA-S-1.4	Probabilistic Risk Assessment Standard for Advanced Non-Light Water Reactor Nuclear Power Plants	2021	ANS	
5	3.14	Process for Infrastructure Aging Management and Life Extension of Nonreactor Nuclear Facilities	2021	ANS	0
6	55.1	Solid Radioactive Waste Processing System for Light-Water-Cooled Reactor Plants	2021	ANS	0
7	56.8	Containment System Leakage Testing Requirements	2020	ANS	0
8	18.1	Radioactive Source Term for Normal Operation of Light Water Reactors	2020	ANS	27
9	2.27	criteria for investigations of nuclear facility sites for seismic hazard assessments	2020	ANS	47
10	2.29	Probabilistic Seismic Hazards Analysis	2020	ANS	67
11	6.1.1	Photon and Neutron Fluence-to-Dose Conversion Coefficients	2020	ANS	19
12	51.10	Auxiliary feedwater system for pressurized water reactors	2020	ANS	41
13	54.1	Nuclear Safety Criteria and Design Process for Sodium Fast Reactor Nuclear Power Plants	2020	ANS	48
14	57.8	Fuel Assembly Identification	2020	ANS	36
15	58.8	Time Response Design Criteria for Safety-Related Operator Actions	2019	ANS	23
16	2.8	Probabilistic Evaluation of External Flood Hazards for Nuclear Facilities	2019	ANS	81
17	8.23	Nuclear Criticality Accident Emergency Planning and Response	2019	ANS	40
18	16.1	Measurement of the leachability of solidified low-level radioactive wastes by a short-term test procedure	2019	ANS	37
19	19.1	nuclear data sets for reactor design calculations	2019	ANS	25
20	19.6.1	Reload Startup Physics Tests for Pressurized Water Reactors	2019	ANS	36
21	3.5	Nuclear Power Plant Simulators for Use in Operator Training and Examination	2018	ANS	35
22	57.3	Design Requirements for New Fuel Storage Facilities at Light Water Reactor Plants	2018	ANS	20
23	2.6	Guidelines for Estimating Present & Projecting Future Population Distributions Surrounding Nuclear Facility Sites	2018	ANS	40
24	2.10	Criteria for Retrieval, Processing, Handling, and Storage of Records from Nuclear Facility Seismic Instrumentation	2017	ANS	28
25	2.15	criteria for modeling and calculating atmospheric dispersion of routine radiological releases from nuclear facilities	2017	ANS	64
26	2.26	categorization of nuclear facility structures, systems, and components for seismic design	2017	ANS	29
27	3.2	American National Standard managerial, administrative, and quality assurance controls for the operational phase of nuclear power plants	2017	ANS	36
28	8.3	Criticality accident alarm system	2017	ANS	26
29	8.5	Use of borosilicate-glass Raschig rings as a neutron absorber in solutions of fissile material	2017	ANS	20
30	8.6	Safety in conducting subcritical neutron-multiplication measurements in situ	2017	ANS	12
31	8.7	Nuclear Criticality Safety in the Storage of Fissile Materials	2017	ANS	28
32	16.1	Measurement of the leachability of solidified low-level radioactive wastes by a short-term test procedure	2017	ANS	40
33	19.3.4	The Determination of Thermal Energy Deposition Rates in Nuclear Reactors	2017	ANS	15
34	19.3	Steady-state neutronics methods for power reactor analysis	2017	ANS	38

35	8.24	validation of neutron transport methods for nuclear criticality safety calculations	2017	ANS	34
36	19.4	Validation of Neutron Transport Methods for Nuclear Criticality Safety Calculations	2017	ANS	34
37	19.11	Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Water Moderated Power Reactors	2017	ANS	36
38	57.8	Fuel Assembly Identification	2017	ANS	16
39	19.11	Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Pressurized Water Reactors	2017	ANS	36
40	55.1	Solid Radioactive Waste Processing System for Light-Water-Cooled Reactor Plants	2017	ANS	52
41	58.8	Time Response Design Criteria for Safety-Related Operator Actions	2017	ANS	22
42	58.14	Safety and Pressure Integrity Classification Criteria for Light Water Reactors	2017	ANS	77
43	56.8	Containment System Leakage Testing Requirements	2016	ANS	42
44	40.37	Mobile low-level radioactive waste processing systems	2016	ANS	49
45	57.10	Design Criteria for Consolidation of LWR Spent Fuel	2016	ANS	46
46	53.1	nuclear safety design process for modular helium-cooled reactor plants	2016	ANS	135
47	2.2	Earthquake Instrumentation Criteria for Nuclear Power Plants	2016	ANS	44
48	2.3	Estimating Tornado, Hurricane, and Extreme Straight Line Wind Characteristics at Nuclear Facility Sites	2016	ANS	19
49	15.4	Selection and training of personnel for research reactors	2016	ANS	24
50	15.11	radiation protection at research reactor facilities	2016	ANS	42
51	18.1	Radioactive Source Term for Normal Operation of Light Water Reactors	2016	ANS	26
52	2.23	Nuclear Plant Response to an Earthquake	2016	ANS	59
53	19.6.1	Reload Startup Physics Tests for Pressurized Water Reactors	2016	ANS	33
54	19.10	Methods for Determining Neutron Fluence in BWR and PWR Pressure Vessel and Reactor Internals	2016	ANS	14
55	15.2	Quality Control for Plate-Type Uranium-Aluminum Fuel Elements	2016	ANS	20
56	8.22	Nuclear Criticality Safety Based on Limiting and Controlling Moderators	2016	ANS	15
57	8.26	Criticality safety engineer training and qualification program	2016	ANS	11
58	10.4	verification and validation of non-safety-related scientific and engineering computer programs for the nuclear industry	2016	ANS	38
59	10.5	Accommodating user needs in scientific and engineering computer software development	2016	ANS	14
60	8.12	nuclear criticality control and safety of plutonium-uranium fuel mixtures outside reactors	2016	ANS	22
61	8.14	use of soluble neutron absorbers in nuclear facilities outside reactors	2016	ANS	11
62	6.4.2	Specification for Radiation Shielding Materials	2016	ANS	20
63	6.4	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power Plants	2016	ANS	96
64	2.27	criteria for investigations of nuclear facility sites for seismic hazard assessments	2016	ANS	33
65	2.29	Probabilistic Seismic Hazards Analysis	2016	ANS	42
66	2.17	evaluation of subsurface radionuclide transport at commercial nuclear power plants	2016	ANS	42
67	2.21	Criteria for Assessing Atmospheric Effects on the Ultimate Heat Sink	2016	ANS	15
68	8.20	Nuclear criticality safety training	2015	ANS	16
69	9001	Quality management systems - Requirements	2015	ANS	41
70	15.16	Emergency planning for research reactors	2015	ANS	21
71	10.8	Non-Real-Time, High-Integrity Software for the Nuclear Industry User Requirements	2015	ANS	31
72	8.27	Burnup Credit for LWR Fuel	2015	ANS	25
73	8.10	Criteria for nuclear criticality safety controls in operations with shielding and confinement	2015	ANS	15
74	6.6.1	Calculation and measurement of direct and scattered gamma radiation from LWR nuclear power plants	2015	ANS	47
75	3.11	Determining Meteorological Information at Nuclear Facilities	2015	ANS	57

76	2.30	Criteria for Assessing Tectonic Surface Fault Rupture and Deformation at Nuclear Facilities	2015	ANS	59
77	58.9	Single failure criteria for light water reactor safety-related fluid systems	2015	ANS	16
78	59.51	Fuel Oil Systems for Safety-Related Emergency Diesel Generators	2015	ANS	20
79	59.52	Lubricating Oil Systems for Safety-Related Emergency Diesel Generators	2015	ANS	18
80	RA-S-1.2 TRIAL USE	(Trial Use) Probabilistic Risk Assessment Standard for Advanced Non-LWR Nuclear Power Plants	2014	ANS	96
81	58.22 TRIAL USE	(Trial Use) Requirements for Low Power and Shutdown Probabilistic Risk Assessment	2014	ANS	299
82	3.1	Selection, Qualification, and Training of Personnel for Nuclear Power Plants	2014	ANS	36
83	5.1	Decay Heat Power in Light Water Reactors	2014	ANS	61
84	8.1	Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors	2014	ANS	26
85	8.15	Nuclear criticality control of special actinide elements	2014	ANS	28
86	8.19	Administrative Practices for Nuclear Criticality Safety	2014	ANS	14
87	58.16	Safety Categorization and Design Criteria for Nonreactor Nuclear Facilities	2014	ANS	51
88	58.22	Requirements for Low Power and Shutdown Probabilistic Risk Assessment	2014	ANS	299
89	RA-S-1.2	Severe Accident Progression and Radiological Release (Level 2) PRA Standard for Nuclear Power Plant Applications for Light Water Reactors (LWRs)	2014	ANS	96
90	8.17	Criticality Safety Criteria for the Handling, Storage, and Transportation of LWR Fuel Outside Reactors	2014	ANS	11
91	14.1	Operation of Fast Pulse Reactors	2014	ANS	13
92	15.1	The development of technical specifications for research reactors	2013	ANS	24
93	15.8	Quality assurance program requirements for research reactors	2013	ANS	20
94	5.10	Airborne Release Fractions at Non-Reactor Nuclear Facilities - Includes Appendix A: 2013, Appendix B and Appendix C	2013	ANS	39
95	2.15	criteria for modeling and calculating atmospheric dispersion of routine radiological releases from nuclear facilities	2013	ANS	64
96	3.4	medical certification and monitoring of personnel requiring operator licenses for nuclear power plants	2013	ANS	44
97	6.1.2	Neutron and Gamma-Ray Cross Sections for Nuclear Radiation Protection Calculations for Nuclear Power Plants	2013	ANS	18
98	10.7	non-real-time, high-integrity software for the nuclear industry-developer requirements	2013	ANS	30
99	RA-S-1.4 TRIAL USE	(Trail Use) Severe Accident Progression and Radiological Release (Level 2) PRA Standard for Nuclear Power Plant Applications for Light Water Reactors (LWRs)	2013	ANS	501
100	15.21	format and content for safety analysis reports for research reactors	2012	ANS	35
101	41.5	verification and validation of radiological data for use in waste management and environmental remediation	2012	ANS	57
102	3.2	American National Standard managerial, administrative, and quality assurance controls for the operational phase of nuclear power plants	2012	ANS	36
103	2.21	Criteria for Assessing Atmospheric Effects on the Ultimate Heat Sink	2012	ANS	15
104	8.23	Nuclear Criticality Accident Emergency Planning and Response	2012	ANS	31
105	19.1	nuclear data sets for reactor design calculations	2011	ANS	17
106	8.21	use of fixed neutron absorbers in nuclear facilities outside reactors	2011	ANS	13
107	58.14	Safety and Pressure Integrity Classification Criteria for Light Water Reactors	2011	ANS	77
108	2.3	Estimating Tornado, Hurricane, and Extreme Straight Line Wind Characteristics at Nuclear Facility Sites	2011	ANS	19
109	5.4	Method for calculating the fractional release of volatile fission products from oxide fuel	2011	ANS	20
110	53.1	nuclear safety design process for modular helium-cooled reactor plants	2011	ANS	135
111	19.3	Steady-state neutronics methods for power reactor analysis	2011	ANS	38
112	19.6.1	Reload Startup Physics Tests for Pressurized Water Reactors	2011	ANS	33

113	2.17	evaluation of subsurface radionuclide transport at commercial nuclear power plants	2010	ANS	42
114	57.5 ERTA	Light Water Reactors Fuel Assembly Mechanical Design and Evaluation	2010	ANS	0
115	10.2	Portability of Scientific and Engineering Software	2009	ANS	15
116	3.5	Nuclear Power Plant Simulators for Use in Operator Training and Examination	2009	ANS	33
117	19.10	Methods for Determining Neutron Fluence in BWR and PWR Pressure Vessel and Reactor Internals	2009	ANS	14
118	40.37	Mobile low-level radioactive waste processing systems	2009	ANS	49
119	15.11	radiation protection at research reactor facilities	2009	ANS	34
120	15.16	Emergency planning for research reactors	2008	ANS	16
121	8.27	Burnup Credit for LWR Fuel	2008	ANS	13
122	10.4	verification and validation of non-safety-related scientific and engineering computer programs for the nuclear industry	2008	ANS	38
123	2.27	criteria for investigations of nuclear facility sites for seismic hazard assessments	2008	ANS	33
124	2.29	Probabilistic Seismic Hazards Analysis	2008	ANS	42
125	58.3	Physical Protection for Nuclear Safety-Related Systems and Components	2008	ANS	47
126	51.10	Auxiliary feedwater system for pressurized water reactors	2008	ANS	33
127	8.23	Nuclear Criticality Accident Emergency Planning and Response	2007	ANS	31
128	8.24	validation of neutron transport methods for nuclear criticality safety calculations	2007	ANS	28
129	8.26	Criticality safety engineer training and qualification program	2007	ANS	11
130	15.4	Selection and training of personnel for research reactors	2007	ANS	18
131	15.1	The development of technical specifications for research reactors	2007	ANS	24
132	6.3.1	Program for Testing Radiation Shields in Light Water Reactors (LWR)	2007	ANS	21
133	10.5	Accommodating user needs in scientific and engineering computer software development	2006	ANS	14
134	6.4.2	Specification for Radiation Shielding Materials	2006	ANS	20
135	6.4	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power Plants	2006	ANS	96
136	8.19	Administrative Practices for Nuclear Criticality Safety	2005	ANS	10
137	3.11	Determining Meteorological Information at Nuclear Facilities	2005	ANS	44
138	5.1 ERTA	Decay Heat Power in Light Water Reactors	2005	ANS	1
139	5.1	Decay Heat Power in Light Water Reactors	2005	ANS	51
140	57.1	Design Requirements for Light Water Reactor Fuel Handling Systems	2005	ANS	22
141	2.26	categorization of nuclear facility structures, systems, and components for seismic design	2004	ANS	29
142	8.17	Criticality Safety Criteria for the Handling, Storage, and Transportation of LWR Fuel Outside Reactors	2004	ANS	13
143	8.14	use of soluble neutron absorbers in nuclear facilities outside reactors	2004	ANS	11
144	14.1	Operation of Fast Pulse Reactors	2004	ANS	15
145	16.1	Measurement of the leachability of solidified low-level radioactive wastes by a short-term test procedure	2003	ANS	42
146	19.1	nuclear data sets for reactor design calculations	2002	ANS	17
147	19.3.4	The Determination of Thermal Energy Deposition Rates in Nuclear Reactors	2002	ANS	18
148	56.8	Containment System Leakage Testing Requirements	2002	ANS	41
149	2.23	Nuclear Plant Response to an Earthquake	2002	ANS	35
150	58.9	Single failure criteria for light water reactor safety-related fluid systems	2002	ANS	16
151	1	Conduct of Critical Experiments	2000	ANS	10
152	10.2	Portability of Scientific and Engineering Software	2000	ANS	15
153	15.2	Quality Control for Plate-Type Uranium-Aluminum Fuel Elements	1999	ANS	18

154	5.10	Airborne Release Fractions at Non-Reactor Nuclear Facilities - Includes Appendix A: 2013, Appendix B and Appendix C	1998	ANS	39
155	8.7	Nuclear Criticality Safety in the Storage of Fissile Materials	1998	ANS	28
156	8.1	Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors	1998	ANS	26
157	59.52	Lubricating Oil Systems for Safety-Related Emergency Diesel Generators	1998	ANS	18
158	59.51	Fuel Oil Systems for Safety-Related Emergency Diesel Generators	1997	ANS	20
159	8.3	Criticality accident alarm system	1997	ANS	30
160	8.22	Nuclear Criticality Safety Based on Limiting and Controlling Moderators	1997	ANS	20
161	19.11	Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Water Moderated Power Reactors	1997	ANS	28
162	8.5	Use of borosilicate-glass Raschig rings as a neutron absorber in solutions of fissile material	1996	ANS	20
163	57.5	Light Water Reactors Fuel Assembly Mechanical Design and Evaluation	1996	ANS	26
164	57.10	Design Criteria for Consolidation of LWR Spent Fuel	1996	ANS	44
165	57.8	Fuel Assembly Identification	1995	ANS	16
166	58.11	Design criteria for safe shutdown following selected design basis events in light water reactors - Addendum:07/23/2002	1995	ANS	19
167	8.21	use of fixed neutron absorbers in nuclear facilities outside reactors	1995	ANS	13
168	15.8	Quality assurance program requirements for research reactors	1995	ANS	20
169	58.8	Time Response Design Criteria for Safety-Related Operator Actions	1994	ANS	23
170	55.4	Gaseous Radioactive Waste Processing Systems for Light Water Reactor Plants	1993	ANS	37
171	57.1	Design Requirements for Light Water Reactor Fuel Handling Systems	1992	ANS	22
172	55.1	Solid Radioactive Waste Processing System for Light-Water-Cooled Reactor Plants	1992	ANS	50
173	58.3	Physical Protection for Nuclear Safety-Related Systems and Components	1992	ANS	47
174	51.10	Auxiliary feedwater system for pressurized water reactors	1991	ANS	33
175	8.20	Nuclear criticality safety training	1991	ANS	16
176	8.12	nuclear criticality control and safety of plutonium-uranium fuel mixtures outside reactors	1987	ANS	28
177	6.6.1	Calculation and measurement of direct and scattered gamma radiation from LWR nuclear power plants	1987	ANS	44
178	6.3.1	Program for Testing Radiation Shields in Light Water Reactors (LWR)	1987	ANS	21
179	8.6	Safety in conducting subcritical neutron-multiplication measurements in situ	1983	ANS	12
180	8.10	Criteria for nuclear criticality safety controls in operations with shielding and confinement	1983	ANS	16
181	8.15	Nuclear criticality control of special actinide elements	1981	ANS	26

Hercules Ebooks Institute

www.herculesebooks.com info@herculesebooks.com +989141908737